



# **Fast Reactors and Related Fuel Cycles: Next Generation Nuclear Systems for Sustainable Development FR17**

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**IAEA**

International Atomic Energy Agency

# Fast Reactor Power Stations Proceedings Of The International Conference

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## **Fast Reactor Power Stations Proceedings Of The International Conference:**

Fast Reactor Power Stations, 1974 Proceedings of an international conference held on 11-14 March 1974 Fast Reactor Power Stations British Nuclear Energy Society, 1974 **Nuclear Science Abstracts**, 1975 **Reactor Technology**, 1969 **Nuclear Safety**, 1978 **Accessions of Unlimited Distribution Reports**, 1973-11-16

*Handbook of Generation IV Nuclear Reactors* Igor Pioro, 2022-12-07 Handbook of Generation IV Nuclear Reactors Second Edition is a fully revised and updated comprehensive resource on the latest research and advances in generation IV nuclear reactor concepts. Editor Igor Pioro and his team of expert contributors have updated every chapter to reflect advances in the field since the first edition published in 2016. The book teaches the reader about available technologies, future prospects, and the feasibility of each concept presented, equipping them with a strong skillset which they can apply to their own work and research. Provides a fully updated, revised, and comprehensive handbook dedicated entirely to generation IV nuclear reactors. Includes new trends and developments since the first publication, as well as brand new case studies and appendices. Covers the latest research, developments, and design information surrounding generation IV nuclear reactors. *AEC Research and Development Report* Atomic Energy Commission, 1970 *WASH*, 1970 Reactor and Fuel-processing Technology, 1967 **Energy Research Abstracts**, 1985 *Sodium Fast Reactors with Closed Fuel Cycle* Baldev Raj, P. Chellapandi, P. R. Vasudeva Rao, 2015-04-15 Sodium Fast Reactors with Closed Fuel Cycle delivers a detailed discussion of an important technology that is being harnessed for commercial energy production in many parts of the world. Presenting the state of the art of sodium-cooled fast reactors with closed fuel cycles, this book offers in-depth coverage of reactor physics, materials, design, and safety. *Fast Reactor System Design* Naoto Kasahara, 2017-03-01 This book describes the fast reactor (FR) as a type of new reactor for nuclear plants currently under research and development. The book targets young researchers and engineers who will be charged with commercializing this new type of reactor to lead to the development of new components and systems for improved plant reliability and economy. This volume also helps readers to understand the methods of integrating the power plant in its entirety from the reactor core to all of the various systems and components and teaches the way of thinking that forms the background of these methods. This background includes the various organizational and management issues that are encountered as projects move forward and will be explored in great detail based on actual design and construction experience with Japan's prototype FR Monju. **Fast Spectrum Reactors** Alan E. Waltar, Donald R. Todd, Pavel V. Tsvetkov, 2011-09-28 This book is a complete update of the classic 1981 FAST BREEDER REACTORS textbook authored by Alan E. Waltar and Albert B. Reynolds, which along with the Russian translation served as a major reference book for fast reactor systems. Major updates include transmutation physics, a key technology to substantially ameliorate issues associated with the storage of high-level nuclear waste, advances in fuels and materials technology including metal fuels and cladding materials capable of high temperature and high burnup, and new approaches to reactor safety including passive

safety technology New chapters on gas cooled and lead cooled fast spectrum reactors are also included Key international experts contributing to the text include Chaim Braun Stanford University Ronald Omberg Pacific Northwest National Laboratory Massimo Salvatores CEA France Baldev Raj Indira Gandhi Center for Atomic Research India John Sackett Argonne National Laboratory Kevan Weaver TerraPower Corporation James Seinicki Argonne National Laboratory Russell Stachowski General Electric Toshikazu Takeda University of Fukui Japan and Yoshitaka Chikazawa Japan Atomic Energy Agency

*Centrifugal Pumps for Sodium Cooled Reactors* Ravindra Kale, B.K. Shreedhar, 2023-11-21 This comprehensive introduction to centrifugal pumps used in sodium-cooled fast reactors discusses the special attributes of centrifugal pumps design features manufacturing requirements instrumentation and operating experience It covers the characteristics of mechanical pumps used as the main coolant pumps in fast reactors Key Features Covers description of pumps in various reactors highlighting the special features of the pumps and providing an overview of futuristic design concepts Discusses the aspects related to the design manufacture testing instrumentation and operating experience of centrifugal sodium pumps Highlights the challenges in centrifugal sodium pump testing Presents topics such as cavitation testing for critical applications and thermodynamic effect on pump cavitation Real life case studies are included for better understanding This book gives a detailed overview of the design manufacture testing and operating experience of the main coolant pumps used in sodium cooled nuclear reactors It further discusses the special type of pumps used in fast reactor power plants to circulate liquid sodium through the core The text examines the challenges in centrifugal sodium pump testing and types of test facilities around the world Real life examples are used to highlight important aspects It is primarily written for senior undergraduate graduate students and academic researchers in the fields such as mechanical engineering nuclear engineering and chemical engineering

**Proceedings of the First International Conference on Structural Mechanics in Reactor Technology**, 1971

**Nuclear Fission Reactors** I. R. Cameron, 2012-12-06 This book is intended to provide an introduction to the basic principles of nuclear fission reactors for advanced undergraduate or graduate students of physics and engineering The presentation is also suitable for physicists or engineers who are entering the nuclear power field without previous experience with nuclear reactors No background knowledge is required beyond that typically acquired in the first two years of an undergraduate program in physics or engineering Throughout the emphasis is on explaining why particular reactor systems have evolved in the way they have without going into great detail about reactor physics or methods of design analysis which are already covered in a number of excellent specialist texts The first two chapters serve as an introduction to the basic physics of the atom and the nucleus and to nuclear fission and the nuclear chain reaction Chapter 3 deals with the fundamentals of nuclear reactor theory covering neutron slowing down and the spatial dependence of the neutron flux in the reactor based on the solution of the diffusion equations The chapter includes a major section on reactor kinetics and control including temperature and void coefficients and xenon poisoning effects in

power reactors Chapter 4 describes various aspects of fuel management and fuel cycles while Chapter 5 considers materials problems for fuel and other constituents of the reactor The processes of heat generation and removal are covered in Chapter 6

6 Comprehensive Nuclear Materials ,2020-07-22 Materials in a nuclear environment are exposed to extreme conditions of radiation temperature and or corrosion and in many cases the combination of these makes the material behavior very different from conventional materials This is evident for the four major technological challenges the nuclear technology domain is facing currently i long term operation of existing Generation II nuclear power plants ii the design of the next generation reactors Generation IV iii the construction of the ITER fusion reactor in Cadarache France iv and the intermediate and final disposal of nuclear waste In order to address these challenges engineers and designers need to know the properties of a wide variety of materials under these conditions and to understand the underlying processes affecting changes in their behavior in order to assess their performance and to determine the limits of operation Comprehensive Nuclear Materials Second Edition Seven Volume Set provides broad ranging validated summaries of all the major topics in the field of nuclear material research for fission as well as fusion reactor systems Attention is given to the fundamental scientific aspects of nuclear materials fuel and structural materials for fission reactors waste materials and materials for fusion reactors The articles are written at a level that allows undergraduate students to understand the material while providing active researchers with a ready reference resource of information Most of the chapters from the first Edition have been revised and updated and a significant number of new topics are covered in completely new material During the ten years between the two editions the challenge for applications of nuclear materials has been significantly impacted by world events public awareness and technological innovation Materials play a key role as enablers of new technologies and we trust that this new edition of Comprehensive Nuclear Materials has captured the key recent developments Critically reviews the major classes and functions of materials supporting the selection assessment validation and engineering of materials in extreme nuclear environments Comprehensive resource for up to date and authoritative information which is not always available elsewhere even in journals Provides an in depth treatment of materials modeling and simulation with a specific focus on nuclear issues Serves as an excellent entry point for students and researchers new to the field Energy Research Abstracts ,1985

**Proceedings of the ANS/ASME/NRC International Topical Meeting on Nuclear Reactor Thermal-Hydraulics: LMFBF and HTGR advanced reactor concepts and analysis methods** ,1980

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